

Journal of RISK AND RELIABILITY

Effects of remedial actions on small piping reliability

Proc IMechE Part O: J Risk and Reliability 227(2) 144–161 © IMechE 2013 Reprints and permissions: sagepub.co.uk/journalsPermissions.nav DOI: 10.1177/1748006X13477798 pio.sagepub.com



Abdelmoumene Guedri

Abstract

This article describes probabilistic calculations that address intergranular stress corrosion cracking of stainless steel piping; a degradation mechanism of major concern to nuclear pressure boundary integrity. The objective is to simulate the cracking of stainless steel piping under intergranular stress corrosion cracking conditions, and to evaluate the structural reliability using remedial actions for intergranular stress corrosion cracking that are limited to benefits of in-service inspections and the induction heating stress improvement process. The results show that an effective in-service inspection requires a suitable combination of flaw detection capability and inspection schedule, and it has been suggested that the residual stresses could be altered to become favorable, thereby improving the reliability piping.

Keywords

Probabilistic fracture mechanics, stress corrosion cracking, structural reliability, in-service inspection, Monte Carlo simulation

Date received: 5 October 2012; accepted: 17 January 2013

Introduction

One of the important degradation mechanisms to be considered for alloyed steels is stress corrosion cracking (SCC). This mechanism causes cracking in the material owing to the combined action of a susceptible material, a tensile stress, and a corrosive environment. In boiler water reactor (BWR) piping, the susceptible material is usually AISI 304 stainless steel in a sensitized condition next to weldments. The susceptibility of this material to intergranular SCC (IGSCC) is owing to chromium carbide precipitation in the grain boundaries, which leaves the regions immediately adjacent to these grain boundaries low in corrosion-resistant chromium.¹ The precipitation occurs most commonly under the thermal conditions encountered during welding. The stress is primarily owing to weld shrinkage during fabrication, and the corrosive environment results from coolant oxygen and low impurity levels according to the operating specifications.²

The purpose of this article is to apply probabilistic fracture mechanics (PFM) to analyze the influence of remedial actions on austenitic stainless steels piping structural reliability. PFM provides a technique for estimating the probability of failure of a structure or one of its components when such failures are considered to occur as the result of the sub-critical and catastrophic growth of an initial crack-like defect. Such techniques are inherently capable of treating the influence of nondestructive inspections.^{3–6} Several articles in the literature^{7–11} addressed the probabilistic failure analysis of components subjected to IGSCC. Failure probabilities of a piping component subjected to IGSCC, including the effects of residual stresses, were computed by Guedri et al.^{12–13} using Monte Carlo simulation (MCS) techniques.

IGSCC in the heat-affected zones of stainless steel welds is much more difficult to detect by ultrasonic testing (UT) inspection techniques. The IGSCC tends to be extremely tight, and is often highly branched at the crack tip. It is also difficult to distinguish between UT echo signals from cracks and from the weld root. Thus it is very hard to detect IGSCC, and even more difficult to determine the depth accurately.¹⁴ As a result, UT inservice inspection (ISI), conducted in accordance with the minimum requirements of Section XI of the ASME boiler and Pressure Vessel Code, tends to be of little value for this problem.

Corresponding author:

Abdelmoumene Guedri, Infra-Res Laboratory, University of Souk Ahras, PO Box 1553, Souk Ahras 41000, Algeria. Email: guedri_moumen@yahoo.fr

Infra-Res Laboratory, University of Souk Ahras, Souk Ahras, Algeria



Figure 19. Effect of reducing residual stress at midlife with IHSI process. IHSI: induction-heating stress improvement.

provide a useful basis to generalize results for pipingleak probabilities. This article has also discussed POD curves and the benefits of ISI in the framework of reductions in the leak probabilities for nuclear piping systems subjected to IGSCC based on $D\sigma$. The results for typical NDE performance levels indicate that low inspection frequencies (one inspection every 10 years) can provide only modest reductions in failure probabilities. More frequent inspections appear to be even more effective. However an "advanced" NDE reliability can achieve a factor of 10 improvements in preventing IGSCC leaks at typical operating conditions even when inspections occur approximately every 10 years; this can be increased to a factor even greater than 10 if the inspection interval is decreased sufficiently. Finally the recommended post-IHSI residual stress has a large effect on reducing the leak probabilities and the lower benefits of ISI for IGSCC can be explained in terms of long incubation periods for stress-corrosion cracking followed by a period of rapid crack growth.

Funding

This research received no specific grant from any funding agency in the public, commercial, or not-for-profit sectors.

Acknowledgements

The author would like to thank Dr Moe Khaleel (PNNL) and Professor Y. Djebbar (Infra-Res Laboratory) for their support during the course of this work.

References

 Zhang S, Shibata T and Haruna T. Initiation and propagation of IGSCC for sensitized type 304 stainless steel in dilute sulfate solutions. *Corrosion Sci* 1997; 39(9): 1725–1739.

- Andresen PL and FP Ford. Fundamental modeling of environment cracking for improved design and lifetime evaluation in BWRs. *Int J Pressure Vessels and Piping* 1994; 59(1–3): 61–70.
- Harris DO, EY Lim and DD Dedhia. Probability of pipe fracture in the primary coolant loop of a PWR plant. Volume 5: *Probabilistic Fracture Mechanics Analysis* – Load Combination Program Project 1 Final Report. NUREG/CR-2189, Vol. 5. Washington, DC: US Nuclear Regulatory Commission, 1981.
- Harris DO, DD Dedhia and ED Eason. Probabilistic Analysis of initiation and growth of stress corrosion cracks in BWR piping. *Am Soc Mech Engrs*, New York 1986: ASME Paper 86-PVP-11.
- Harris DO, Dedhia DD, Eason ED, et al. Probability of failure in BWR reactor coolant piping: probabilistic treatment of stress corrosion cracking in 304 and 316NG BWR piping weldments. *NUREG/CR-4792*, Vol. 3. Washington, DC: US Nuclear Regulatory Commission, 1986.
- You J-S and Wu W-F. Probabilistic failure analysis of nuclear piping with empirical study of Taiwan's BWR plants. *Int J Pressure Vessels and Piping* 2002; 79: 483–492.
- Ting K. The evaluation of intergranular stress corrosion cracking problems of stainless steel piping in Taiwan BWR-6 nuclear power plant. *Nuclear Engng Des* 1999; 191(2): 245–254.
- Rahman S. A computer model for probabilistic leak-rate analysis of nuclear piping and piping welds. *Int J Pressure Vessels and Piping* 1997; 70: 209–221.
- Helie M, Peyrat C, Raquet G, et al. Phenomenological modelling of stress corrosion cracking. Inter. corr. /96 First Global Internet Corrosion Conferences, 1996.
- Lu BT, Chen ZT, Luo JL, et al. Pitting and stress corrosion cracking behaviour in welded austenitic stainless steel. *Electrochimica Acta* 2005; 50(6): 1391–1403.
- Anoop MB, Balaji Rao K and Lakshmanan N. Safety assessment of austenitic steel nuclear power plant pipelines against stress corrosion cracking in the presence of hybrid uncertainties. *Int J Pressure Vessels and Piping* 2008; 85(4): 238–247

- Guedri A, Zeghloul A and Merzoug B. Reliability analysis of BWR piping including the effect of residual Stresses. *Int Rev Mech Engng (IREME)* 2009; 3(5): 640–645.
- Guedri A, Merzoug B, Khaleel M, et al. Reliability analysis of low alloy ferritic piping materials. *Damage and fracture mechanics. failure analysis of engineering materials and structures.* Netherlands: Springer, 2009, pp.33–42.
- Kupperman DS, Reimann KJ and Ellingson WA. Evaluation of Ultrasonic Techniques for Detection of Stress-Corrosion Cracks in Stainless Steel Piping. Electric Power Research Institute Report EPRI NP-761, Palo Alto, California, June 1978.
- Harris DO and Dedhia DD. Theoretical and user's manual for PC-PRAISE. A probabilistic fracture mechanics computer code for piping reliability analysis. Washington, DC: US Nuclear Regulatory Commission, 1992, NUREG/ CR-5864, UCRL-ID-109798.
- Priya C, Rao KB, Anoop MB, et al. Probabilistic failure analysis of austenitic nuclear pipelines against stress corrosion cracking. *Proc IMechE, Part C: J Mechanical Engineering Science* 2005; 219(7): 607–626.
- American Society of Materials. ASM handbook: fatigue and fracture. *Materials Information Society International*. USA: American Society of Materials, 1996.
- BS 7910. Guidance on methods of assessing the acceptability of flaws in metallic structures. London, UK: British Standard Institution, 1999, ch.7.
- R6. Assessment of the integrity of structures containing defects. Gloucester: British Energy Generation Ltd, 2001, Revision 4, chapter I and II.3.
- ASME Boiler and pressure vessel design code, Section XI. Philadelphia: American Society of Mechanical Engineers, 1992.
- Leek TH and Howard IC. An examination of methods of assessing interacting surface cracks by comparison with experimental data. *Int J Pressure Vessels and Piping* 1996; 68: 181–201.
- Leek T and Howard I. Rules for the assessment of interacting surface cracks under Mode I load. *Int J Pressure Vessels and Piping* 1994; 60: 323–339.
- Howard I and Leek T. Estimating the elastic interaction factors of two coplanar surface cracks under Mode I load. *Int J Pressure Vessels and Piping* 1994; 60: 307–321.
- Shi P and Mahadevan S. Corrosion fatigue and multiple site damage reliability analysis. *Int J Fatigue* 2003; 25: 457–469.
- Pitt S and Jones R. Multiple-site and widespread fatigue damage in aging aircrafts. *Engng Failure Analysis* 1997; 4: 237–257.
- 26. Herrera ML, Mattson RA, Tang SS et al. Probabilistic fracture mechanics analysis to justify in-service inspection intervals for the Helms penstock field welds. Proceedings of Waterpower '99-hydro's future: technology, markets and policy, Las Vegas, Nevada, United States, American Society of Civil Engineers, July 6–9, 1999.
- Doctor SR, Becker FL, Heasler PG, et al. Effectiveness of U.S. inservice inspection technologies: a round robin test. In: *Proceedings of a specialist meeting on defect detection and sizing* (CSNI Report No. 75 and EUR 9066 II EN), 1983, 2, 669–678.
- Taylor TT, Spanner JC, Heasler PG, et al. An evaluation of human reliability in ultrasonic inservice inspection for intergranular stress corrosion cracking through round robin testing. *Mater Eval* 1989; 47: 338.

- Heasler PG and Doctor SR. Piping inspection round robin. NUREG/CR-5068, PNNL-10475.U.S. Washington, DC: Nuclear Regulatory Commission, 1996.
- Khaleel MA and Simonen FA. Effects of alternative inspection strategies on piping reliability. *Nuc Engng Des* 2000; 197: 115–140.
- Harris DO, DD Dedhia, ED Eason, et al. Probabilistic treatment of stress corrosion cracking in sensitized 304 stainless steel weldments in BWR piping. *Failure Analysis* Associates Report to Lawrence Livermore National Laboratory. Livermore, California, 1985.
- 32. Khaleel MA, Simonen FA, Harris DO, et al. The impact of inspection on intergranular stress corrosion cracking for stainless steel piping. *Risk and safety assessment: where is the balance*. ASME PVP, 1995, Vol. 266/SERA-Vol. 3, 411–422.
- Khaleel MA and Simonen FA. Evaluations of structural failure probabilities and candidate inservice inspection programs. NUREG/CR-6986; PNNL-13810, Pacific Northwest National Laboratory, Richland, WA, 2009.
- Failure Analysis Associates. PRAISE enhancements to include general strain hardening exponents and mid-life residual stress and water chemistry changes. FaAA-SF-R-90-06-06, CRL-CR-105339. Prepared for Lawrence Livermore National Laboratory, Livermore, California, 1990.
- Bishop BA. An updated structural reliability model for piping risk informed ISI, In: *Fatigue and Fracture*. New York: American Society of Mechanical Engineers, 1997, Vol. 2, 346, 245–252.
- Brickstad B, OJV Chapman, T Schimpfke, et al. Review and Benchmarking of SRM and Associated Software. *NURBIM Final Report* D4, Contract FIKS-CT-2001-00172. Stockholm, Sweden: DNV, 2004.
- Brickstad B and W Zang. NURBIT nuclear RBI analysis tool, a software for risk management of nuclear components. *Technical Report No 10334900-1*. Stockholm, Sweden: DNV, 2001.
- Bell CD and Chapman OJV. Description of PRODI-GAL. NURBIM Report D4/Appendix F. Derby, UK: Rolls-Royce plc, 2003.
- Schimpfke T. A short description of the piping reliability code PROST. NURBIM Report D4/Appendix C. Berlin, Germany: Gesellschaft fur Anlagen-und Reaktorsicherheit (GRS), 2003.
- Dillstrom P. A short description of ProSACC. NURBIM Report D4/Appendix G. Stockholm, Sweden: DNV, 2003.
- Mohammed AA. A Short Description of STRUEL. *NURBIM Report D4/Appendix H.* Cambridge, UK: The Welding Institute, 2003.

Appendix I

Notation

a	crack depth
A_{0-6}	coefficients computed from the table
	given in ASM Handbook
A_{cr}	area of crack
A_p	area of cross-section of pipe
b	one-half of crack length
$C_{I} - C_{9}$	material dependent constants
C_{12}, C_{13}, C_{15}	material dependent constants